

Gamma Spectrometry of Shielded Sources

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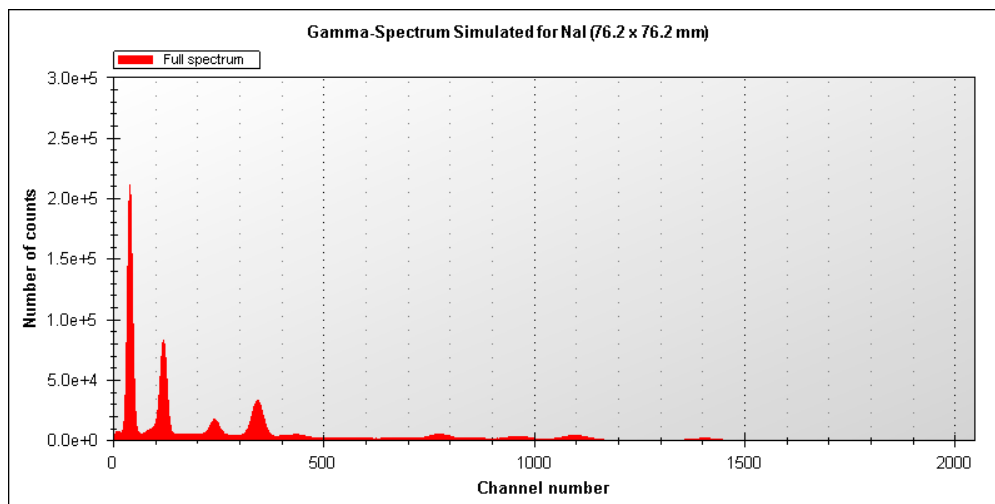
<http://itu.jrc.ec.europa.eu/>

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Outline

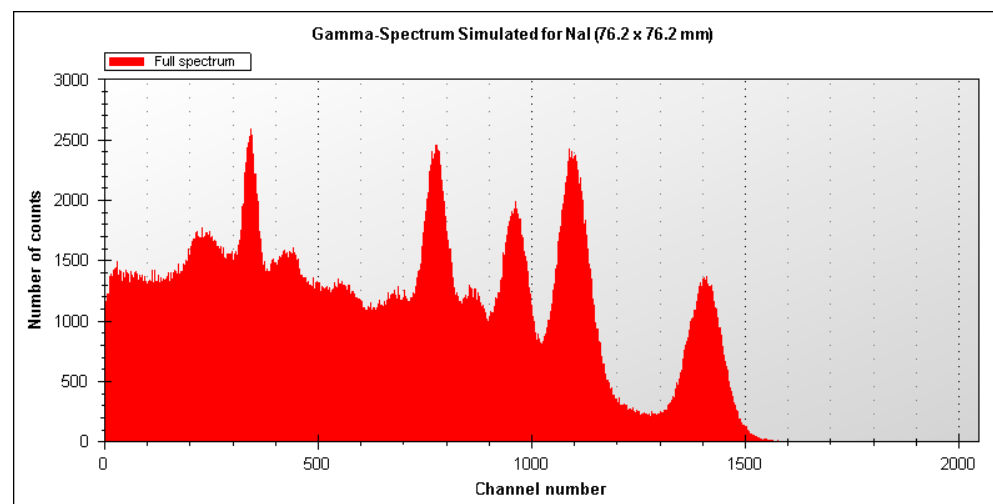
- **Theoretical considerations**
- **Gamma Spectrum Simulator**
- **Live demonstration**

Unshielded

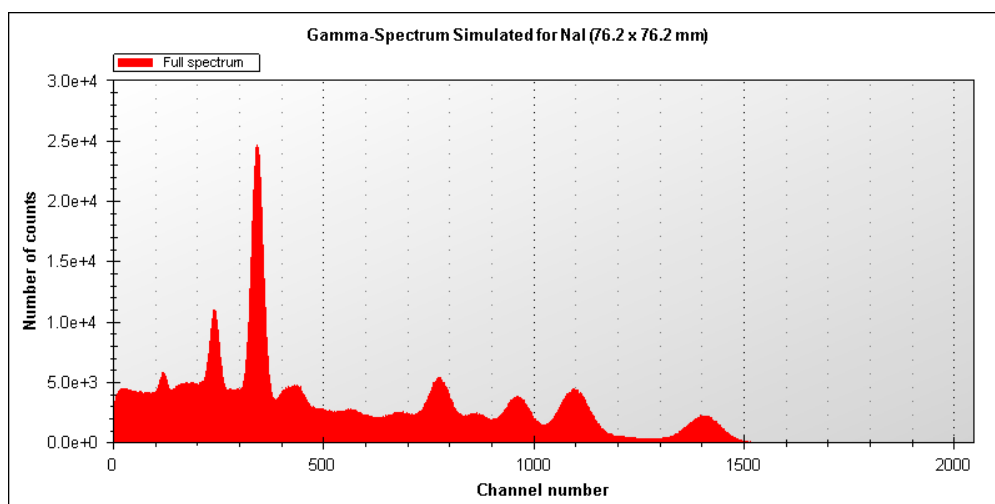


**NaI (3"x3"), Eu-152, 1 MBq, 1000 s,
25 cm from detector, point source**

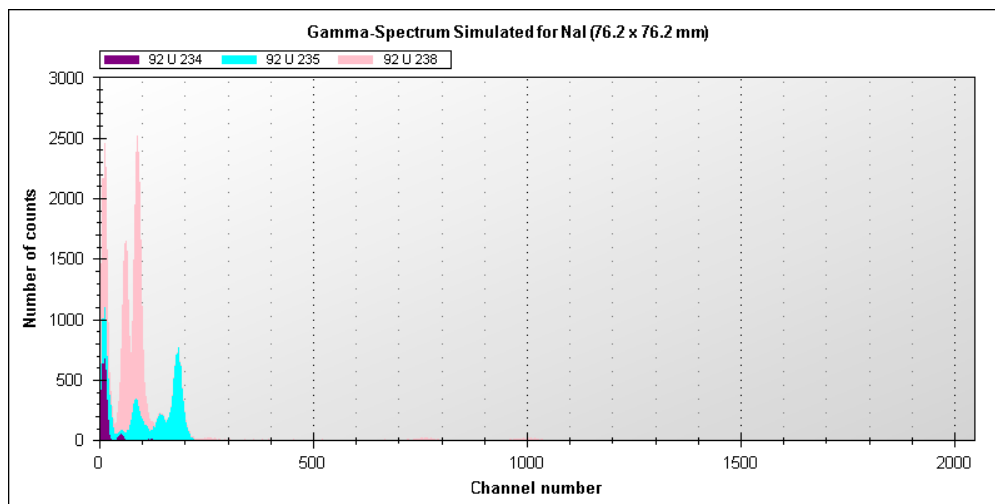
Shielded with 10 mm lead



Shielded with 1 mm lead

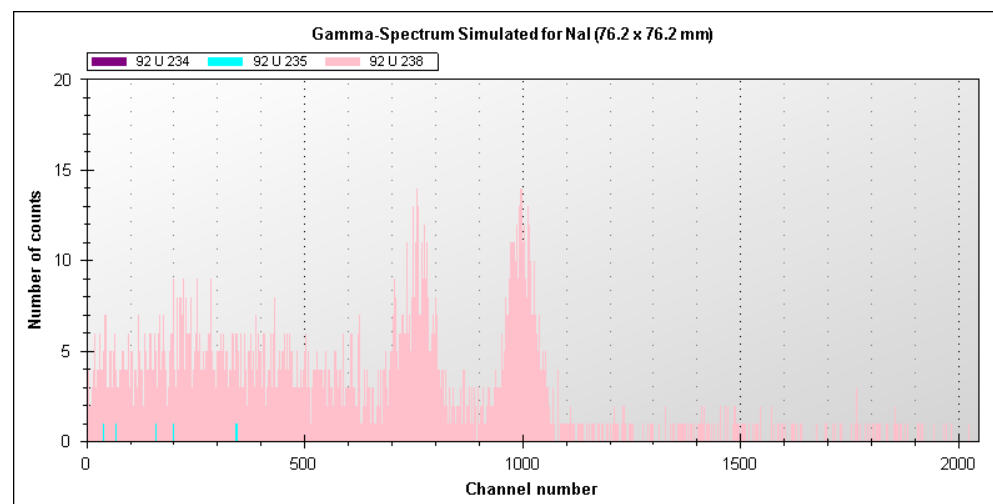


Unshielded

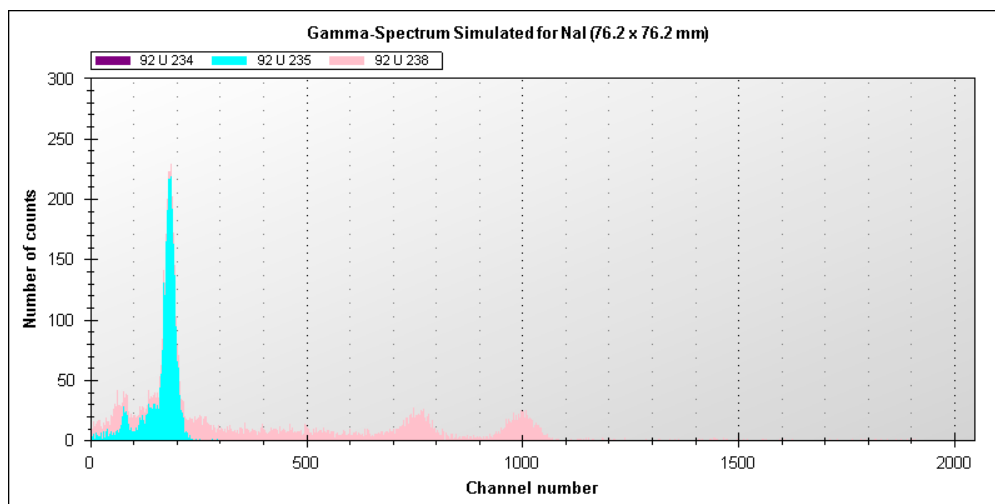


**NaI (3"x3"), NatU 1g, 10000 s,
25 cm from detector, point source**

Shielded with 10 mm lead



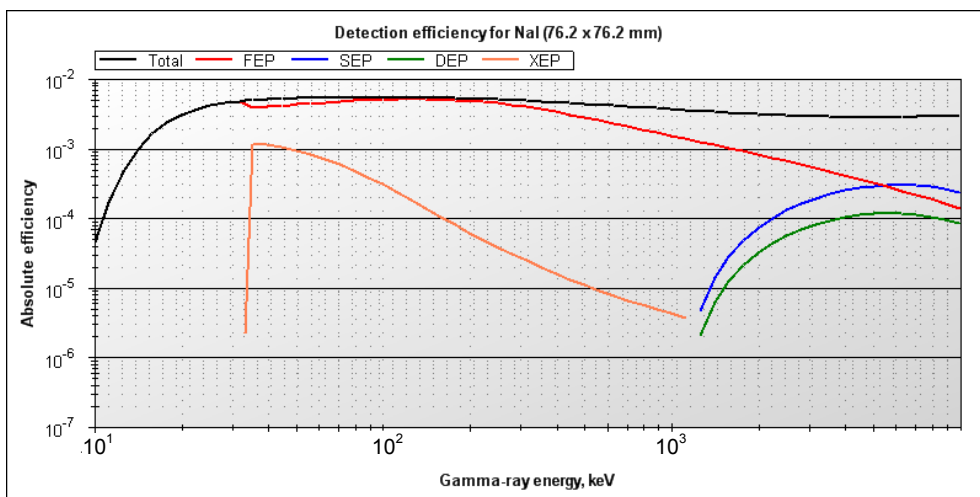
Shielded with 1 mm lead



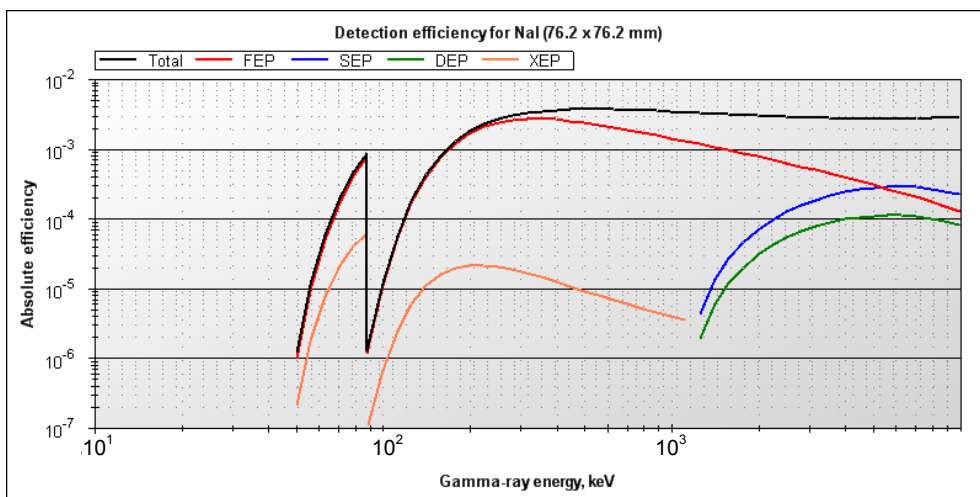
Basic effects of shielding:

- Spectrum shape changes drastically (low-energy lines disappear, high-energy lines start to give major contribution to the spectrum),
- Count rate can significantly decrease => influence of background => detectability considerations,
- Signatures of some nuclides can completely disappear (loss of information) => this is the worst case because it is not possible to predict characteristics of the source based on such measurements.

Unshielded

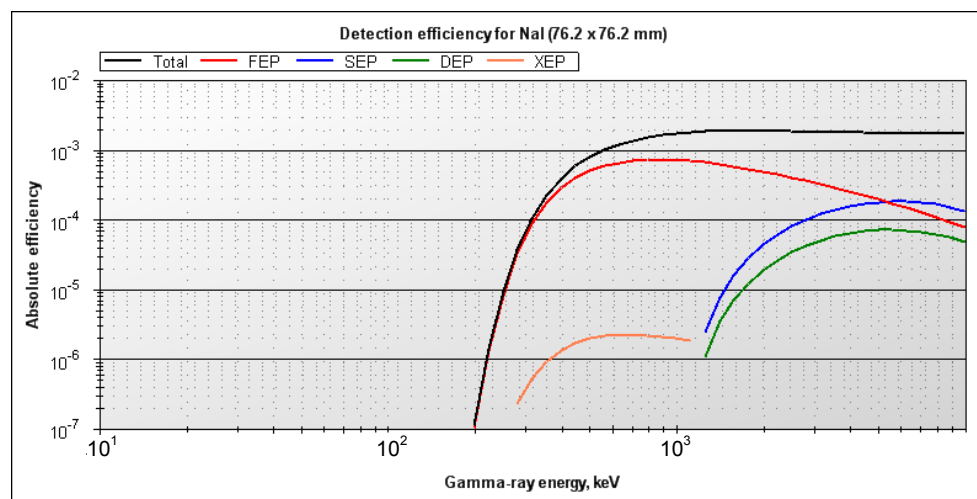


Shielded with 1 mm lead



Detection efficiency of NaI (3"x3")

Shielded with 10 mm lead



Correction for the attenuation in shielding

The full energy peak (FEP) count rate in the case of

unshielded source:

$$I_U = S_U / t_U = A \gamma \varepsilon_U$$

S – the area of the full energy peak,
 t – the measurement time interval,
 A – the activity of a source.

shielded source:

$$I_S = S_S / t_S = A \gamma \varepsilon_S$$

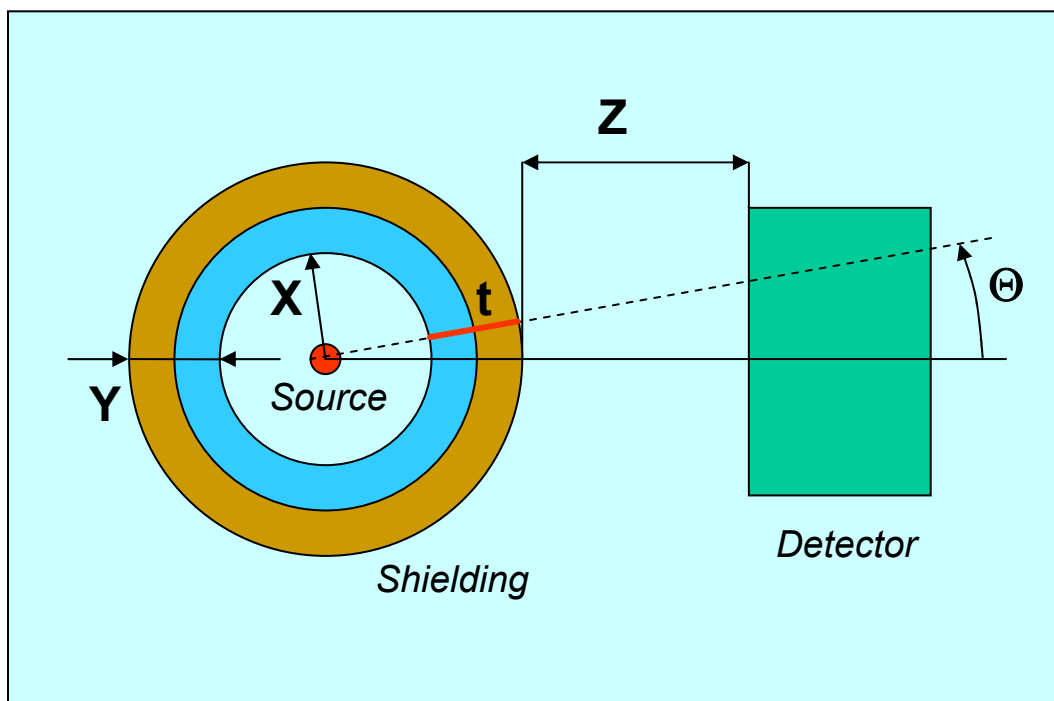
ε – the FEP detection efficiency,
 γ – the gamma-ray emission probability,

The unshielded peak count rate can be derived from the measurement of a shielded source as:

$$I_U = K I_S$$

where $K = \varepsilon_U / \varepsilon_S$ is the energy dependent attenuation correction factor.

Isotropic point source and spherical shielding



$$K = \exp(\mu_{abs} Y)$$

$t = Y$ - the photon pathway in the shielding (cm) - **does not depend on the angle Θ !**

μ_{abs} - photon attenuation coefficient (1/cm) describing the absorption of photons in the shielding

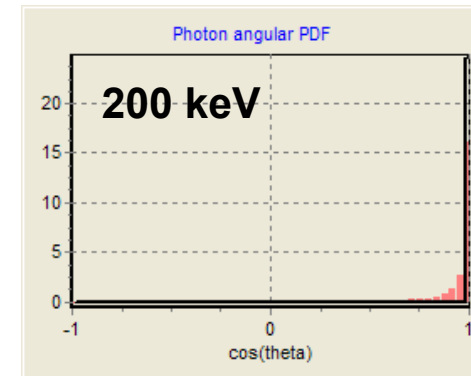
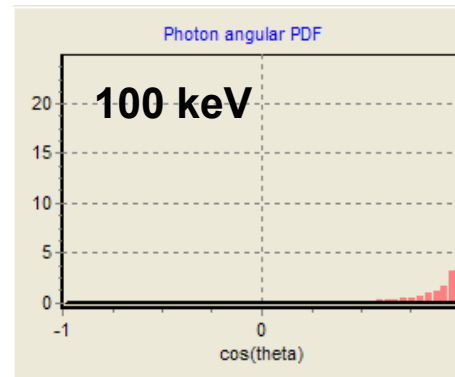
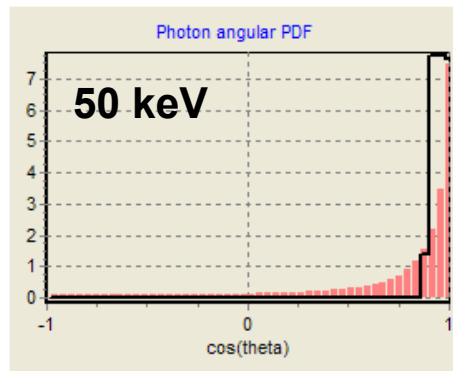
For multiple layer shielding: $\mu_{abs} Y = \sum_i (\mu_{abs} Y)_i$ - the sum over all shielding layers

Is μ_{abs} equal to μ_{tot} ?

$$\mu_{\text{tot}} = \mu_{\text{photoeffect}} + \mu_{\text{compton scattering}} + \mu_{\text{pair production}} + \mu_{\text{coherent scattering}}$$

Coherent scattering = Rayleigh scattering – the incident photon is absorbed by the atomic shell and re-emitted in the other direction **without changing its energy**.

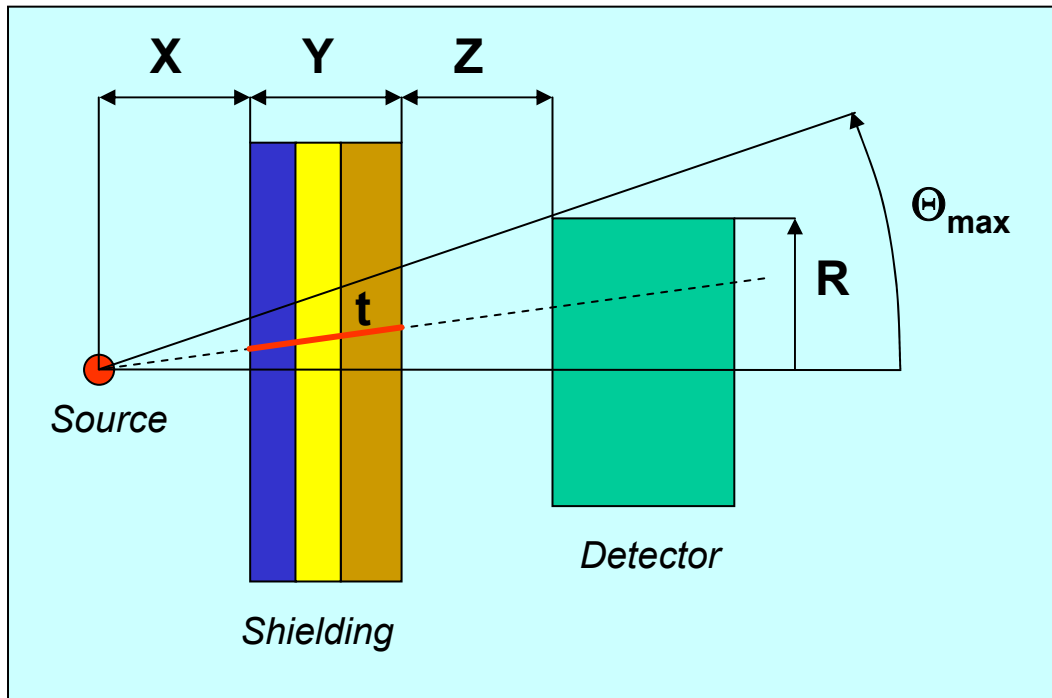
Coherent scattering angular distribution Probability Density Functions (PDF) for Pb:



Normally better results are obtained by assuming that:

Approximation !!! $\mu_{\text{abs}} = \mu_{\text{tot}}$ for $E_{\gamma} < 100 \text{ keV}$
 $\mu_{\text{abs}} = \mu_{\text{tot}} - \mu_{\text{coherent scattering}}$ for $E_{\gamma} > 100 \text{ keV}$ **Approximation !!!**

Isotropic point source and slab shielding



$$K = \frac{\int_{\cos \Theta_{max}}^1 \varepsilon_0(\Theta) d \cos \Theta}{\int_{\cos \Theta_{max}}^1 \varepsilon_0(\Theta) \exp(-\mu_{abs} t(\Theta)) d \cos \Theta}$$

$$\cos \Theta_{max} = \frac{X + Y + Z}{\sqrt{R^2 + (X + Y + Z)^2}}$$

$$t(\Theta) = \frac{Y}{\cos \Theta} \quad \text{- the thickness of shielding at angle } \Theta$$

$\varepsilon_0(\Theta)$ - the FEP efficiency for a pencil photon beam striking the detector at angle Θ

Small detectors or/and large source-to-detector distances: $X + Y + Z \gg R$ (i.e. $\cos \Theta_{max} \approx 1$)

Underestimation !!!

$$K \approx \exp(\mu_{abs} Y)$$

Underestimation !!!

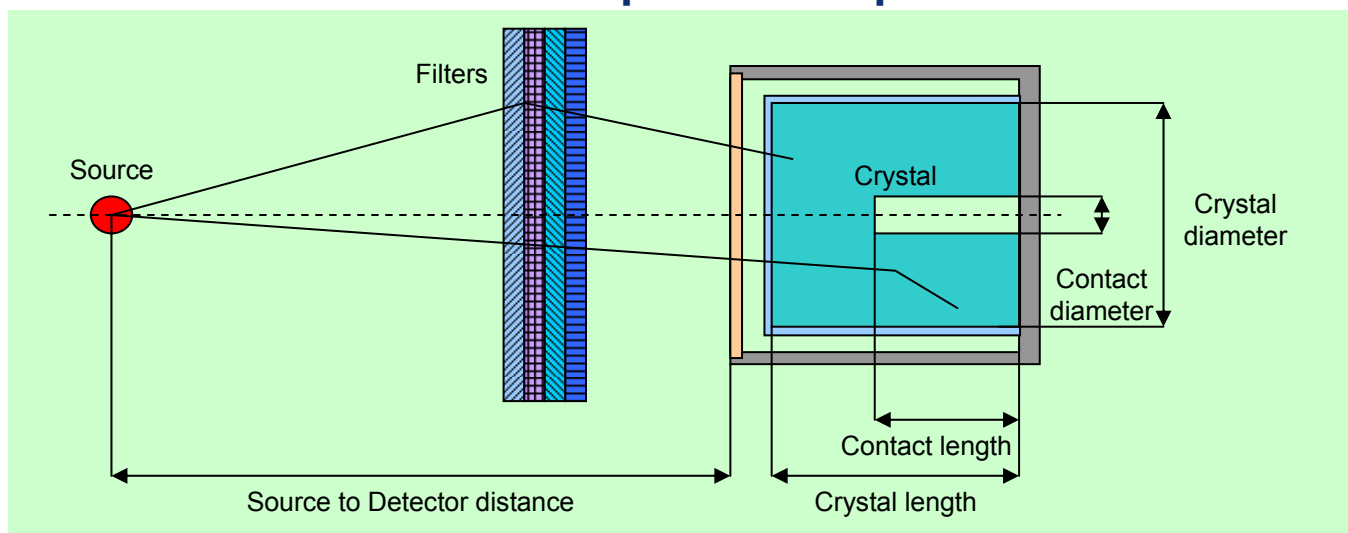
Case study: Ø5x5 cm NaI and Pb slab shielding

E_γ , keV	Y, cm	Integration with $\mu_{\text{abs}} = \mu_{\text{tot}} - \mu_{\text{coh}}$		$K = \exp(\mu_{\text{abs}} Y)$	
		X+Y+Z = 5 cm	X+Y+Z = 50 cm	$\mu_{\text{abs}} = \mu_{\text{tot}} - \mu_{\text{coh}}$	$\mu_{\text{abs}} = \mu_{\text{tot}}$
100	0.1	588.5	438.3	423.7	538.0
	1.0	$1.54 \cdot 10^{27}$	$2.61 \cdot 10^{26}$	$1.86 \cdot 10^{26}$	$2.03 \cdot 10^{27}$
	5.0	$3.01 \cdot 10^{133}$	$1.22 \cdot 10^{132}$	$2.20 \cdot 10^{131}$	$3.50 \cdot 10^{136}$
500	0.1	1.19	1.19	1.19	1.20
	1.0	5.81	5.46	5.44	6.16
	5.0	6510	4850	4780	8880
1000	0.1	1.08	1.08	1.08	1.08
	1.0	2.22	2.16	2.16	2.23
	5.0	54.3	47.5	47.6	55.0

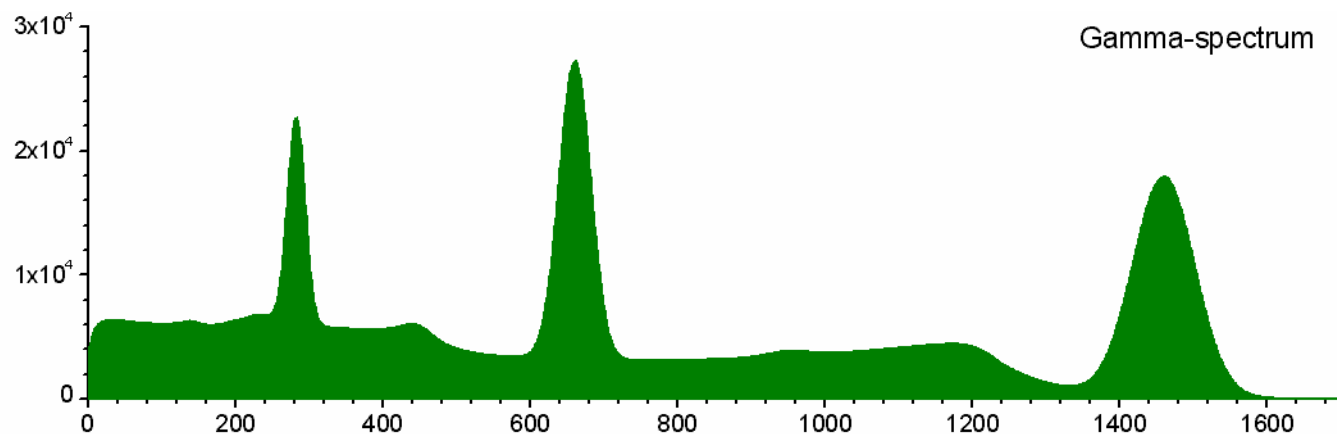
Conclusions (slab shielding)

- The use of exact formula(s) for K is highly recommended when accurate evaluation of the activity of a shielded source is required.
- In the far measurement geometries the approximation $K = \exp(\mu_{\text{abs}} Y)$ with $\mu_{\text{abs}} = \mu_{\text{tot}} - \mu_{\text{coh}}$ is quite accurate. In the close geometries it gives underestimated values of the shielding correction factor.
- Even though it is not entirely correct, the approximation $K = \exp(\mu_{\text{abs}} Y)$ with $\mu_{\text{abs}} = \mu_{\text{tot}}$ is more preferable in close measurement geometries as it conservatively overestimates the attenuation properties of a shield.
- General recommendation: One should avoid, if possible, measurements of shielded sources in close geometries.

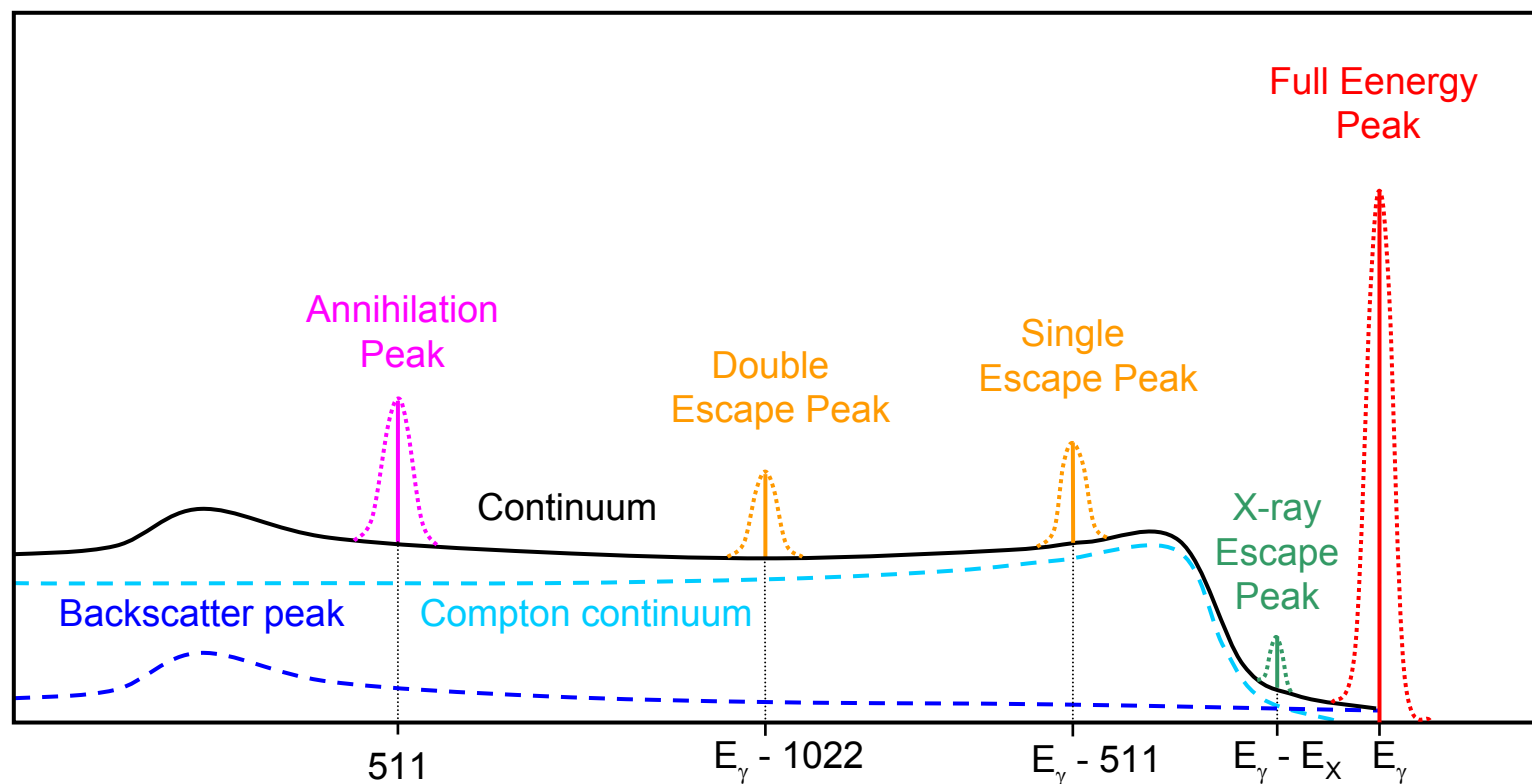
Measurement setup model implemented:



Spectrum modeling procedure:



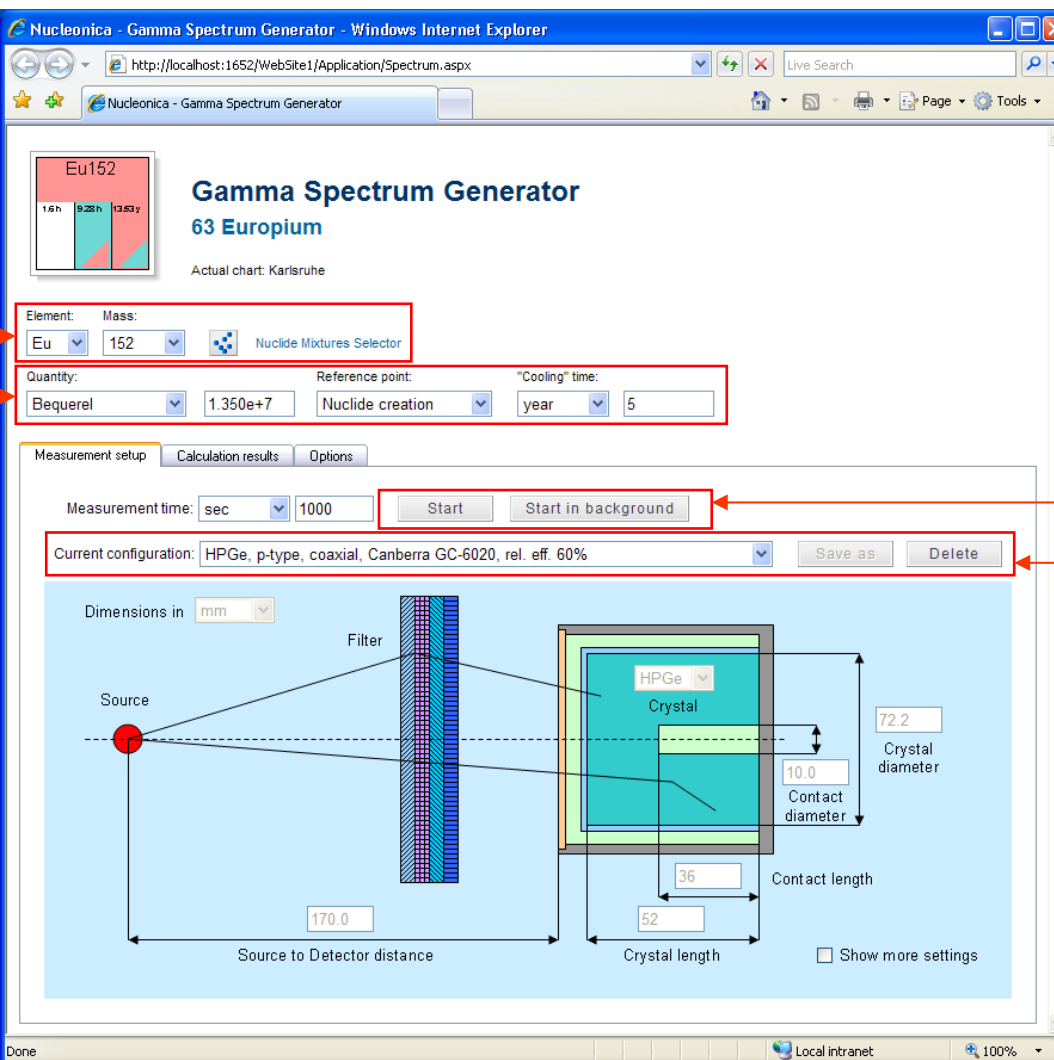
Detector response profile model:



Features implemented: Measurement setup

An arbitrary individual nuclide or a pre-defined mixture of nuclides can be selected as a radiation source

The quantity (activity, mass or number of atoms) of a nuclide or a mixture can be specified either at the moment of its production or at the spectrum measurement starting point of time. In the former case controls for specifying duration of a source cooling time interval become available.



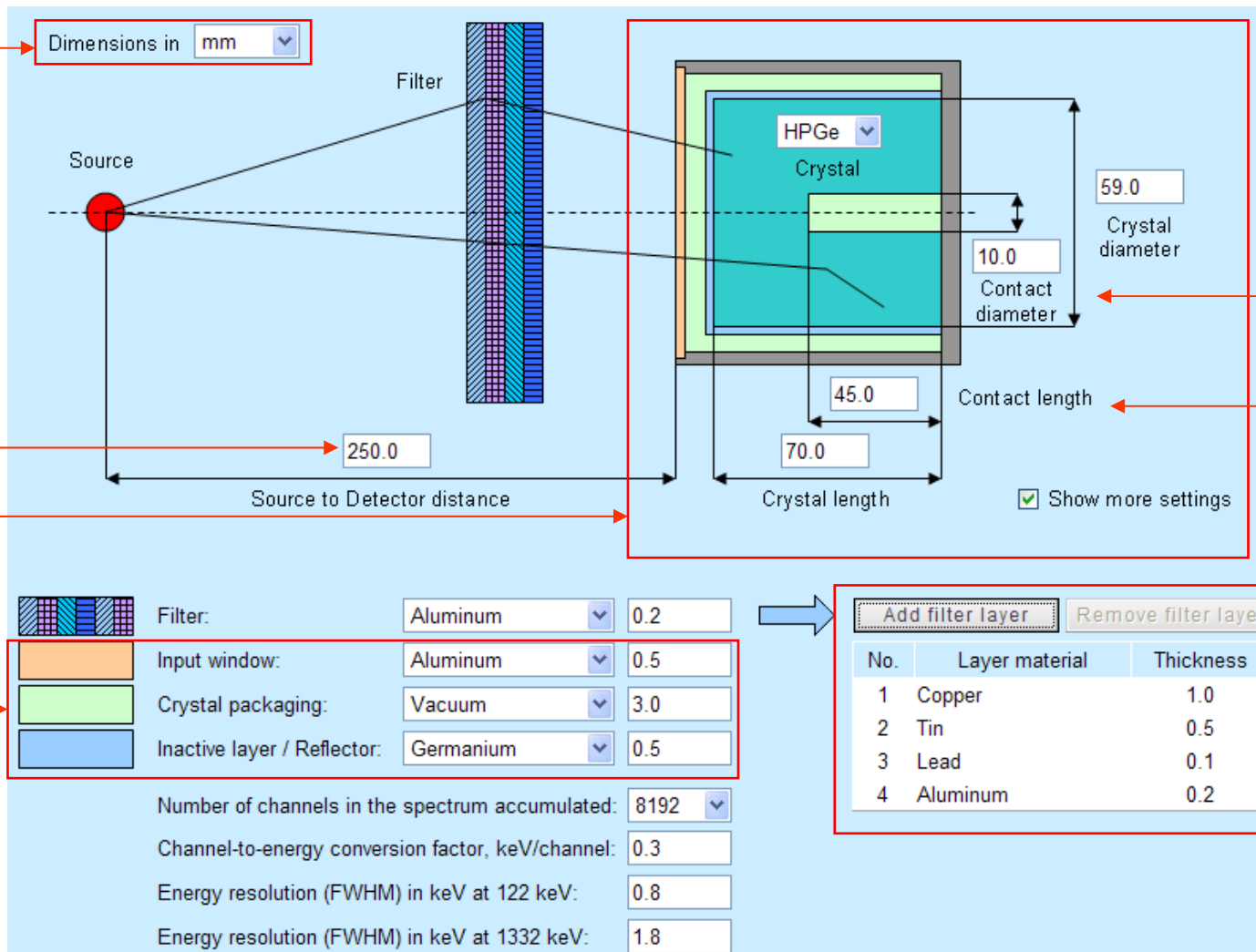
Calculations can be started on-line or in a background mode

A suitable γ -spectrometer can be chosen from 6 pre-defined configurations, which include 2 coaxial HPGe (50% and 150%) detectors, low-energy (LEGe) and broad-energy (BEGe) HPGe detectors, and 2 NaI detectors ($\varnothing 3'' \times 3''$ and $\varnothing 2'' \times 1''$). In addition, user's specific configurations can be managed.

Features implemented: Measurement setup

Dimensions can be entered in "mm", "cm" or "inch" units

The configurable parameters include the source-to-detector distance, as well as dimensions and materials of the detector construction elements.



Dimensions in

Source

Filter

HPGe

Crystal

59.0

Crystal diameter

10.0

Contact diameter

45.0

Contact length

70.0

Crystal length

250.0

Source to Detector distance

☒ Show more settings

Filter:

Input window:

Crystal packaging:

Inactive layer / Reflector:

Number of channels in the spectrum accumulated:

Channel-to-energy conversion factor, keV/channel:

Energy resolution (FWHM) in keV at 122 keV:

Energy resolution (FWHM) in keV at 1332 keV:

No.	Layer material	Thickness
1	Copper	1.0
2	Tin	0.5
3	Lead	0.1
4	Aluminum	0.2

The dimensions of a cylindrical contact at the rear side of the crystal (a construction feature of conventional coaxial HPGe detectors) can be specified


Up to 6 additional absorbing filters made of Al, Cu, Fe, Pb, Sn, or polyethylene can be placed between source and detector

Features implemented: Options

Nucleonica - Gamma Spectrum Generator - Windows Internet Explorer

http://localhost:1652/WebSite1/Application/Spectrum.aspx#

Nucleonica - Gamma Spectrum Generator

 **Gamma Spectrum Generator**
Natural Uranium
Actual chart: Karlsruhe

Nuclide Mixtures:
Natural Uranium Nuclide Selector

Total activity:
Bequerel 2.557e+004

Reference point:
Measurement start

Measurement setup Calculation results Options

Gamma Spectrum Generator Settings:

- ☒ Display detector efficiency curves
- ☒ Consider decay transformations during cooling and counting time intervals
 - ☒ Include gamma-rays of daughter nuclides
 - 0.01 Decay Engine's accuracy factor
- ☒ Consider effects of backscatter radiation
 - 1.0 Backscatter peak normalisation factor

Done Local intranet 100%

Efficiency Graph
can be activated
in the Calculation
Results output

The backscatter
peak simulation
can be switched
on/off, and its
contribution to
the spectrum can
be adjusted

Decay
calculations can
be enabled that
will allow
contributions
from decay
products, being
accumulated
during source
cooling and
spectrum
measurement
time intervals

Features implemented: Calculation results

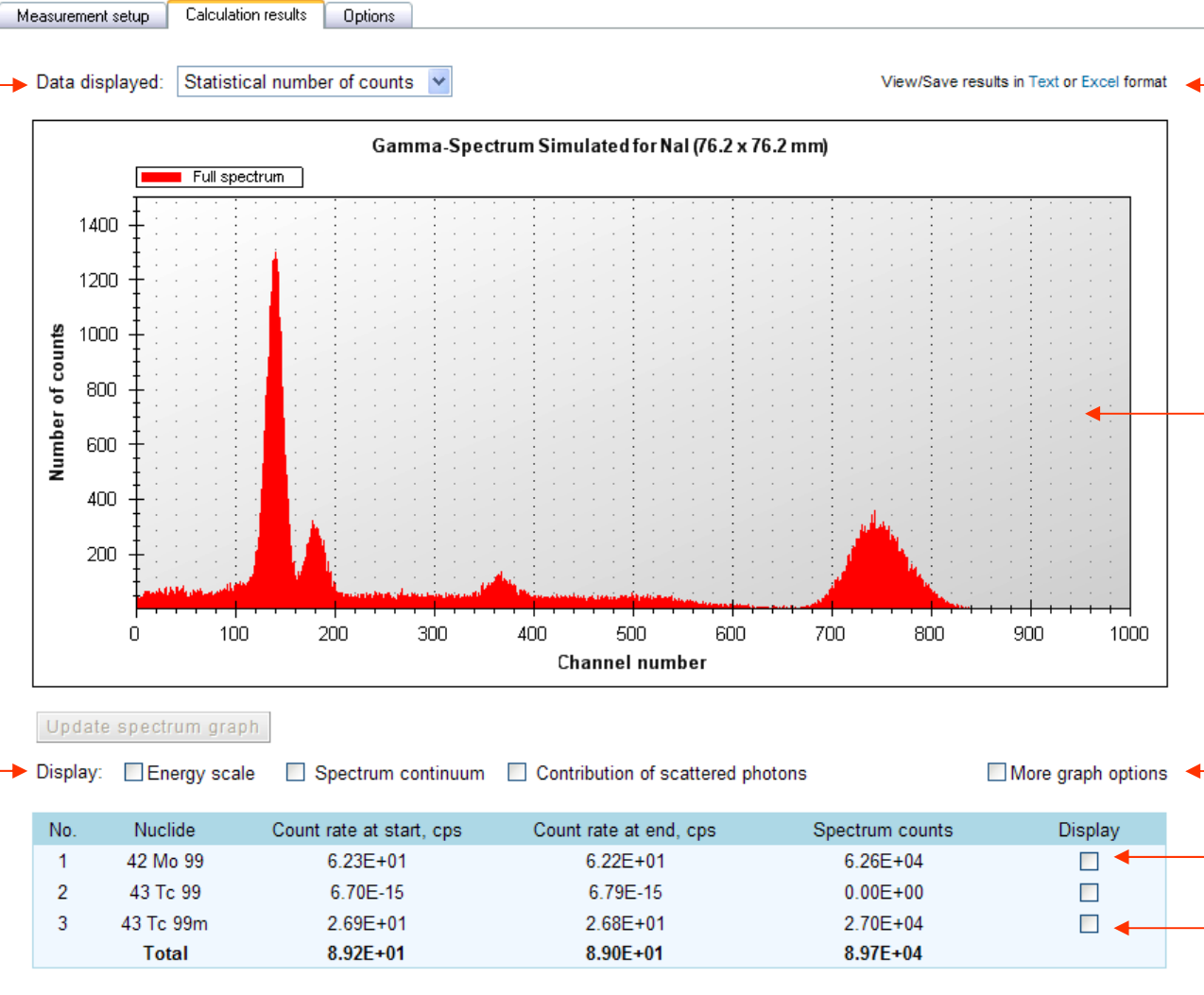
Statistical number of counts

Count rate at start

Count rate at end

Theoretical number of counts

Statistical number of counts



Complete set of spectral information can be downloaded as a text or Excel spreadsheet file

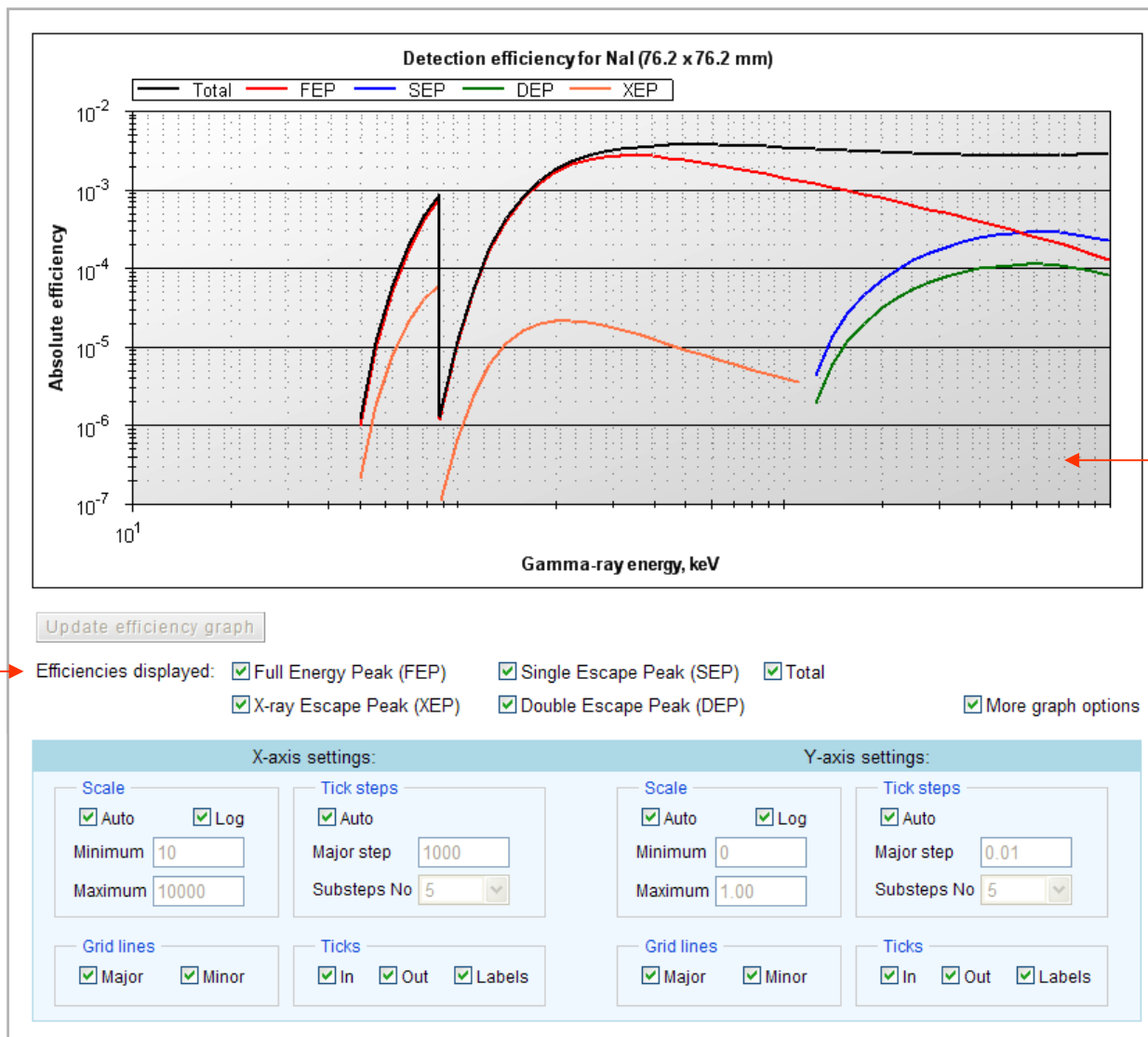
Right click within the graph area enables a context menu, from which one can print or download the spectrum graph

Additional options allow to customize appearance of the graph to meet one's needs and requirements

Display nuclide specific contributions to the full spectrum

No.	Nuclide	Count rate at start, cps	Count rate at end, cps	Spectrum counts	Display
1	42 Mo 99	6.23E+01	6.22E+01	6.26E+04	<input type="checkbox"/>
2	43 Tc 99	6.70E-15	6.79E-15	0.00E+00	<input type="checkbox"/>
3	43 Tc 99m	2.69E+01	2.68E+01	2.70E+04	<input type="checkbox"/>
Total		8.92E+01	8.90E+01	8.97E+04	

Features implemented: Calculation results



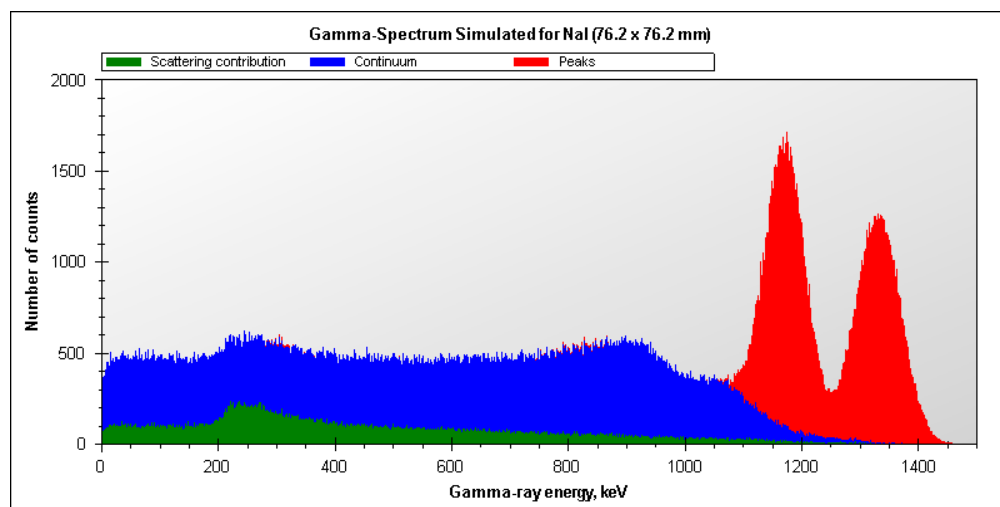
Select efficiency data to be displayed on the graph

Right click within the graph area enables a context menu, from which one can print or download the efficiency graph

Additional options allow to tailor the efficiency graph to one's needs and requirements

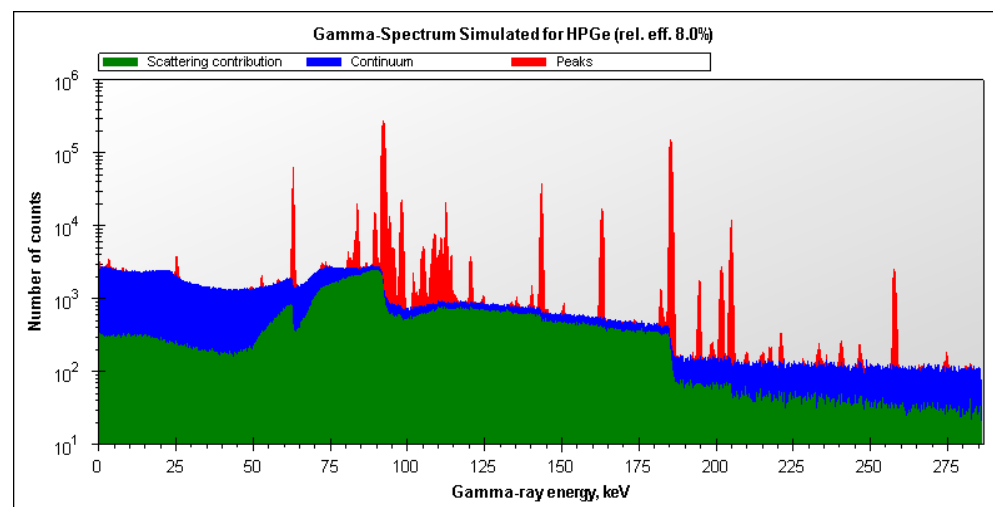
Examples:

100 kBq ^{60}Co



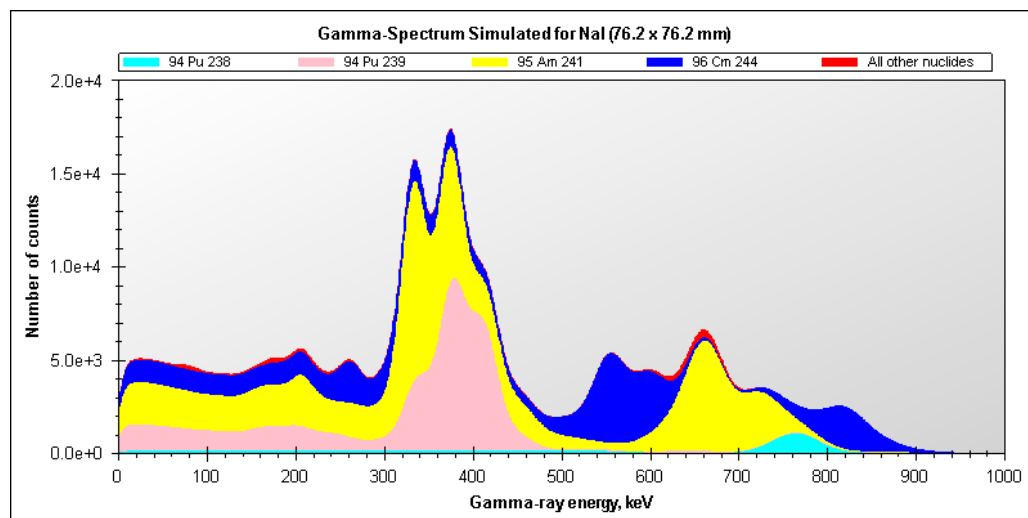
Detector - NaI ($\varnothing 3'' \times 3''$)
Source-to-detector distance - 25 cm
Measurement time - 1000 s

1 g Nat U (2 years after separation)

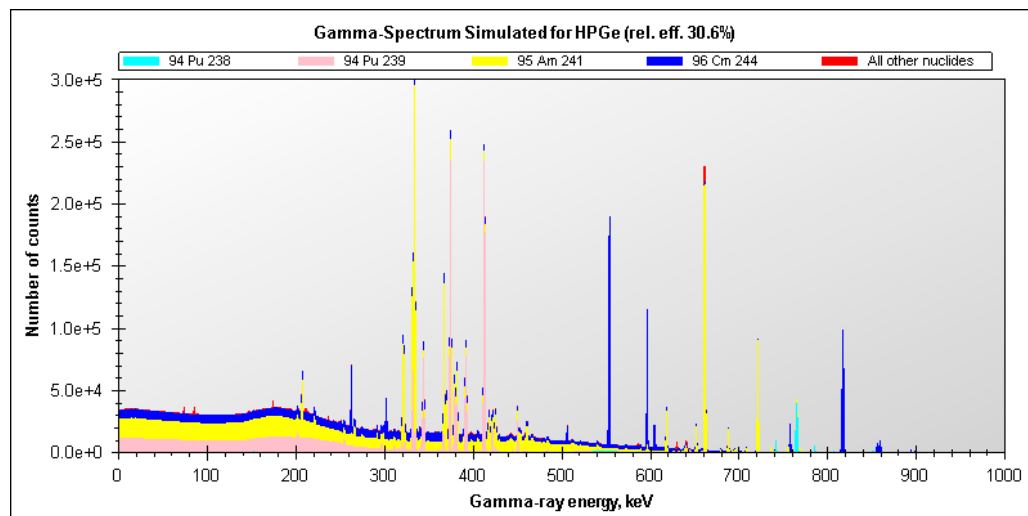


Detector – LEGe (20 mm x 2800 mm²)
Source-to-detector distance – 25 mm
Filter – 0.5 mm Sn
Measurement time - 10⁵ s

Examples:



Detector – NaI (Ø3"×3")
Source-to-detector distance – 25 cm
Filter – 5 mm Pb
Measurement time - 1000 s

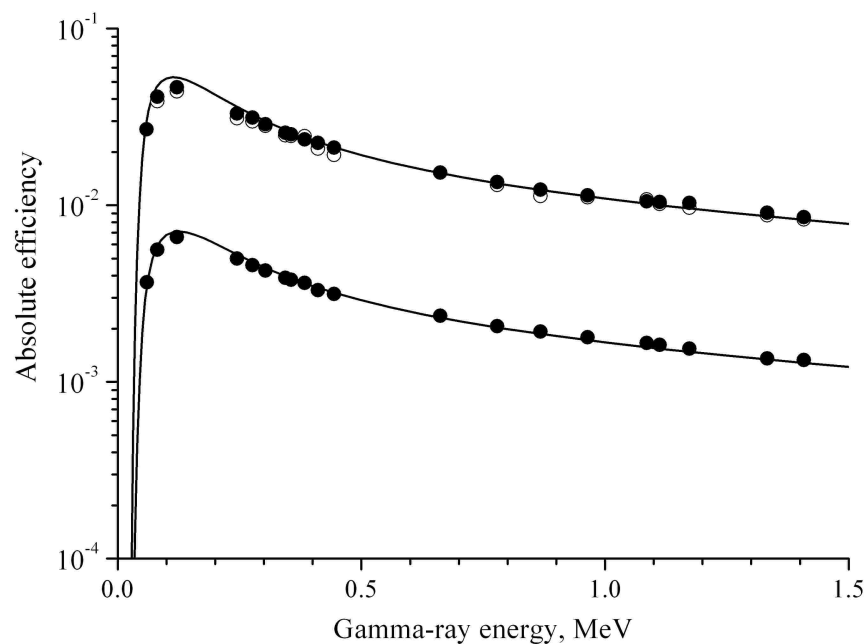


Actinides extracted from 1 kg 6-year-aged PWR spent fuel. Activity - 5.25 TBq

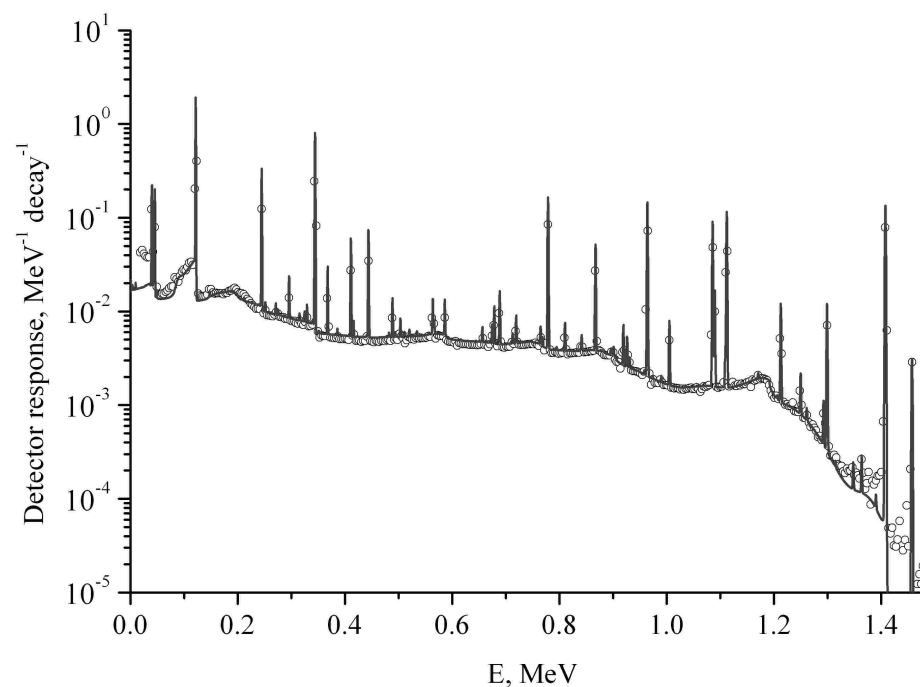
Detector – BEGe (30% rel. eff.)
Source-to-detector distance – 25 cm
Filter – 5 mm Pb
Measurement time - 1000 s

Example:

Results of the experimental validation with 60% HPGe coaxial detector



Full Energy Peak efficiency as a function of the photon energy: circles – experimental values, curve – calculated. Two sets of data refer to the source location at 5 cm and 17 cm distances from the detector end cap.



Calculated (curve) and experimental (circles) detector responses for ^{152}Eu source at 17 cm distance from the detector end cap.

Future work:

- Include simulation of the spectrum distortion effects (e.g. due to coincidence summing and energy resolution deterioration), which may appear in measurements involving elevated count rates and small source-to-detector distances.
- Extend the detector response profile database to include LaBr_3 scintillators that, because of their much superior energy resolution, start to replace traditional NaI crystals in many applications.
- Include self-attenuation effects, which would allow more realistic simulation of gamma-spectra from voluminous sources.

Gamma Spectrum Generator

Thanks for your attention !

